its Supercomputing Centers Program considering the changing nature of computing and information science and technology. Its scope will be limited to NSF's support for advanced computational science.

Agenda: Deliberation of the Draft Report.

Reason for Late Notice: Difficulty with scheduling meeting location.

Dated: May 30, 1995.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 95–13555 Filed 6–1–95; 8:45 am]

BILLING CODE 7555–01–M

Advisory Committee for Social, Behavioral and Economic Sciences; Subcommittee on Transformations to Quality Organizations; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92–463, as amended), the National Science Foundation announces the following meeting:

Name: Advisory Committee for Social, Behavioral and Economic Sciences; Subcommittee on Transformations to Quality Organizations (#1171)

Date & Time: June 21, 1995, 8:30 am–3:30 pm.

Place: Rm 320, National Science Foundation, 4201 Wilson Blvd., Arlington VA.

Type of Meeting: Open.

Contact Person: Dr. Marietta Baba, Program Director, Transformations to Quality Organizations Program, National Science Foundation, 4201 Wilson Boulevard, Room 910, Arlington, VA 22230, 703/306–1757, x7210.

Minutes: May be obtained from the contact person listed above.

Meeting Purpose: To provide advice, recommendations, and oversight concerning support for research, education, and human resources in the areas of the social, behavioral, and economic sciences. To identify preliminary plans to advance Transformations to Quality Organizations (TQO) effort.

Agenda

1. Welcome and Introductions
2. Opening Remarks
3. Importance of Research
4. Role of Subcommittee
5. Current Status of Program
6. Assessment of Year One
7. Work Plan for Year Two
8. Next Steps

Dated: May 30, 1995.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 95–13550 Filed 6–1–95; 8:45 am]

BILLING CODE 7555–01–M

NUCLEAR REGULATORY COMMISSION

Proposed Generic Letter; Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of opportunity for public comment.

SUMMARY: The Nuclear Regulatory Commission (NRC) is proposing to issue a generic letter. This draft generic letter would allow licensees to voluntarily relocate the pressure temperature limit curves and low temperature overpressure protection system limits from the technical specifications to a licensee-controlled document. The NRC is seeking comment from interested parties regarding both the technical and regulatory aspects of the proposed generic letter presented under the Supplementary Information heading. This proposed generic letter and supporting documentation were sent to the Committee to Review Generic Requirements (CRGR). CRGR will review the proposed generic letter after resolution and incorporation of public comments. Relevant information, including model technical specifications and a model safety evaluation, that was sent to the CRGR is available in the Public Document Rooms under accession number 9505220128. The NRC will consider comments received from interested parties in the final evaluation of the proposed generic letter. The NRC's final evaluation will include a review of the technical position and, when appropriate, an analysis of the value/impact on licensees. Should this generic letter be issued by the NRC, it will become available for public inspection in the Public Document Rooms.

DATES: Comment period expires July 3, 1995. Comments submitted after this date will be considered if it is practical to do so, but assurance of consideration cannot be given except for comments received on or before this date.

ADDRESSES: Submit written comments to Chief, Rules Review and Directives Branch, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Written comments may also be delivered to 11545 Rockville Pike, Rockville, Maryland, from 7:30 am to 4:15 pm, Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, 2120 L Street NW. (Lower Level), Washington, DC.

FOR FURTHER INFORMATION CONTACT: Maggalean W. Weston, Technical Specifications Branch, Division of Project Support, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555–001, Telephone (301) 415–3151.

SUPPLEMENTARY INFORMATION:

NRC Generic Letter 95–XX: Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits

Addressees

All holders of operating licenses or construction permits for nuclear power reactors.

Purpose

The U.S. Nuclear Regulatory Commission (NRC) is issuing this generic letter to advise licensees that they may request a license amendment to relocate the pressure temperature (P/T) limit curves and low temperature overpressure protection (LTOP) system limits from the technical specifications (TS) to a pressure temperature limits report (PTLR) or a similar document.

Description of Circumstances

During the development of the improved standard technical specifications (STS), a change was proposed to relocate the P/T curves and LTOP setpoint curves and values currently contained in the TS to a licensee-controlled document. As part of the improvements to the STS, the NRC staff agreed with the industry that the curves and setpoints may be relocated outside the TS where these limits could be maintained more efficiently and at a lower cost to the licensee, provided the parameters for constructing the curves and setpoints are derived using a methodology approved by the NRC.

Discussion

Technical specifications include limiting conditions for operation (LCOs) that establish P/T and LTOP system limits for the reactor coolant system. The limits are defined by figures and values that provide an acceptable range of operating temperatures and pressures for heatup, cooldown, low temperature overpressure, criticality, and inservice leak and hydrostatic testing conditions. These parameters are generally valid for a specified number of effective full-power years.

License amendments are generally required at the end of the effective period for P/T limit curves or when surveillance specimens are withdrawn

and tested. Also, each time the curves are revised, the LTOP system must be reevaluated to ensure that its functional requirements can still be met using relief valves or other methods. Processing amendment requests for changes to TS that are developed using an accepted methodology is an unnecessary burden on licensee and NRC resources. An alternative approach for the control of these limits was proposed during the development of the improved STS. This approach, like that used for the core operating limits, would relocate the P/T curves and the LTOP setpoint curves or values to a PTLR or a similar document and reference that document in the affected LCOs and bases.

The methodology for determining P/T and LTOP system limit parameters must comply with the specific requirements of Appendices G and H to Part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR), be documented in an NRC-approved topical report or in a plant-specific submittal, and be incorporated by reference into the TS. As such, subsequent changes in the methodology must be approved by a license amendment.

Requested Information

Licensees and applicants who voluntarily choose to adopt this line item improvement are encouraged to propose changes that are consistent with the attached guidance. The guidance requires that the licensee be able to reference a methodology for developing the curves and setpoints that has been approved by the NRC, develop a PTLR or a similar document that contains the figures, values, parameters, and explanations derived from the methodology, and make appropriate changes to the applicable sections of the TS. The NRC project manager for the facility will review the amendment requests that conform to the guidance in this generic letter and coordinate the appropriate staff review of the methodology proposed for calculating the P/T limit curves and the LTOP system limits. Amendment requests that do not conform to the guidance in this generic letter will require additional review time.

Required Response

Licensees and applicants who voluntarily choose to adopt this line item improvement should submit a response to the requested information described above.

Backfit Discussion

Any action by licensees to propose TS changes in accordance with the guidance of this generic letter is voluntary and, therefore, not a backfit analysis.

Guidance for a Proposed License Amendment To Relocate the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits

Introduction

This generic letter provides guidance for preparing a license amendment request to modify the technical specifications (TS) to relocate the pressure temperature (P/T) limit curves and low temperature overpressure protection (LTOP) system limits currently contained in the TS to a pressure temperature limits report (PTLR) or a similar document. This alternative was based on a change included in the improved standard technical specifications (STS) to remove the P/T limit curves and LTOP system limits from the TS and relocate them to a PTLR or a similar document to reduce the number of amendment requests associated with changes to the P/T limit curves and LTOP system limits. Since an amendment request must be submitted whenever a change is made to the TS, the relocation of the P/T curves and LTOP system limits will result in a resource savings for the licensees and the NRC by eliminating unnecessary license amendment requests for changes to the P/T limit curves and LTOP system limits in TS when surveillance specimens are withdrawn and tested and additional vessel toughness data become available. To relocate the P/T curves and LTOP system limits from the TS, the licensee must be able to reference a methodology approved by the NRC for deriving the parameters used for constructing the curves and setpoints, develop a PTLR or a similar document, and make appropriate changes to the applicable sections of the TS.

In evaluating the relocation, the NRC staff concluded that, while it is essential to safety to operate the plant within the bounds of P/T limits and to satisfy the regulations that ensure the integrity of the reactor coolant pressure boundary (RCPB), the periodic adjustment of those limits to account for time-dependent parametric changes could be calculated in accordance with a methodology approved by the NRC. Criterion 2 of the Commission's final policy on TS improvements, which was published in the **Federal Register** (58 FR 39132) on July 22, 1993, requires that the TS include operating restrictions (pressure/temperature limits) needed to preclude unanalyzed accidents and transients. However, once the methodology is approved, the licensee may modify the figures, values, and parameters without the need for a license amendment and without affecting nuclear safety, provided these changes are determined using the approved methodology and are consistent with all applicable limits of the plant design assumptions as stated in the FSAR. Additionally, the licensee must submit to the NRC a formal PTLR or a similar document containing the figures, values, and parameters derived from the application of the methodology approved by the NRC. This reporting requirement augments a reporting requirement that is already in effect. Section III of Appendix H currently requires a summary technical report of data relating to capsule withdrawal and specimen test results. Application of these results will also be included in the PTLR. This report will allow the NRC staff to continue monitoring the status of the structural integrity of the reactor vessel even though prior NRC approval of the changes to these limits would not be required if they do not involve an unreviewed safety question.

A new provision was also added to the administrative controls section of the TS indicating that the figures, values, and parameters for inclusion in the PTLR will be verified after each reactor vessel fluence period or when surveillance specimens are withdrawn and tested and a report submitted to the NRC. Hence, the staff can confirm proper application of the methodology approved by the NRC. Further, the PTLR will be referenced in the TS so that the same degree of control on plant operation will be maintained. As a result, this alternative provides the same assurance of compliance with design specifications as before, yet removes the unnecessary burden on both plant and NRC staff of processing amendment requests.

Discussion

Technical specifications include limiting conditions for operation (LCOs) that establish P/T limits for the RCS. This system is designed to withstand the effects of cyclic loads resulting from system temperature and pressure changes. These cyclic loads are introduced by normal load transients, reactor trips, startup and shutdown operations, and hydrostatic and leak rate tests. During startup and shutdown, the rates of temperature and pressure changes are limited so that the maximum specified heatup and cooldown rates are consistent with the design assumptions and satisfy operating limits that provide a wide

margin of safety to brittle failure of the reactor vessel. The P/T limits are periodically modified as the reactor vessel material toughness decreases as a result of material embrittlement caused by neutron irradiation. The periodic modifications are necessary when the applicable effective full-power years (EFPYs) for the P/T limits contained in the TS are about to expire or the reactor vessel material surveillance data indicate an increase in the nil-ductility transition reference temperature (RT_{NDT}).

As required by Appendix G to Part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR), operating P/T limits are calculated and adhered to by plant operations personnel to ensure that fracture toughness requirements for Part 50, specimens of reactor vessel material are installed near the inside reactor vessel wall and are withdrawn on a schedule to provide data on the effects of radiation fluence and the thermal environment on the vessel material. These data are used to adjust the P/T limits, as necessary, to compensate for the shift in material transition temperature as indicated by tests on the withdrawn specimens. The withdrawal and analysis of the specimens and resulting revision of the P/T limit curves make up the requirements necessary to compensate for the shift in material transition temperature. This ensures that the reactor vessel is operated at high enough temperatures to preclude brittle fracture of the vessel material.

The LTOP system controls RCS pressure at low temperatures so that the integrity of the RCPB is not compromised by violating the P/T limits of Appendix G to 10 CFR part 50. The LTOP system provides overpressure protection by limiting coolant input capability and having adequate pressure relief capacity. Each time the P/T limit curves are revised, the LTOP system must be reevaluated to ensure that its functional requirements can still be met. The LTOP system for pressure relief typically consists of two power-operated relief valves (PORVs), two residual heat removal (RHR) suction relief valves, or a combination of both. Some plants have only one PORV. The LTOP system limits consist of PORV and RHR setpoints. The RHR suction relief valves do not have variable pressure and temperature lift setpoints like the PORVs and, therefore, are still addressed in the TS. As designed for the LTOP system, each PORV is signaled to open if the RCS pressure approaches a limit determined by the LTOP system actuation logic. This logic monitors both RCS temperature and RCS pressure to determine when a condition not acceptable in the PTLR is approached. The PORV setpoints should be included in the PTLR and updated when the revised P/T limits conflict with the LTOP system limits. LTOP requirements do not apply to boiling water reactors.

Requirements for Relocating the Curves and Setpoints

Relocation of the curves and setpoints to a licensee-controlled document requires three separate licensee actions. The licensee must (1) have a methodology approved by the NRC to reference in its TS; (2) develop a report such as a PTLR or a similar document to contain the figures, values, parameters, and any explanation necessary; and (3) modify the applicable sections of the TS accordingly.

•*Methodology and PTLR*

The first two of the three requirements for relocating the P/T curves and LTOP system limits are an NRC-approved methodology and the associated reporting requirements in the PTLR. The methodology will consist of only those methods used for calculation, not the calculations themselves. The PTLR will consist of the explanations, figures, values, and parameters derived from the calculations. Since the PTLR will be provided to the NRC upon issuance after each fluence period and after approval of the methodology, a preliminary or draft PTLR should be provided when the methodology is submitted so that questions regarding the content and format can be addressed prior to its formal completion.

The following table shows the relationship between the provisions specified in the STS for the approved methodology and the requirements to be included in the methodology and the PTLR. The provisions for the methodology are those shown in the administrative controls section of the STS.

REQUIREMENTS FOR METHODOLOGY AND PTLR

Provisions for methodology from administrative controls section in sts	Minimum requirements to be included in methodology	Minimum requirements to be included in PTLR
1. The methodology shall describe how the neutron fluence is calculated (reference new Regulatory Guide when issued).	Describe transport calculation methods including computer codes and formulas used to calculate neutron fluence. Provide references.	Provide the values of neutron fluences that are used in the adjusted reference temperature (ART) calculation.
2. The Reactor Vessel Material Surveillance Program shall comply with Appendix H to 10 CFR Part 50. The reactor vessel material irradiation surveillance specimen removal schedule shall be provided, along with how the specimen examinations shall be used to update the PTLR curves.	Briefly describe the surveillance program. Licensee transmittal letter should identify by title and number report containing the Reactor Vessel Surveillance Program and surveillance capsule reports. Topical/generic report contains placeholder only. Reference Appendix H to 10 CFR Part 50.	Provide the surveillance capsule withdrawal schedule, or reference by title and number the documents where the schedule is located. Reference the surveillance capsule reports by title and number if ARTs are calculated using surveillance data.
3. Low temperature overpressure protection (LTOP) system lift setting limits developed using NRC-approved methodologies may be included in the PTLR.	Describe how the LTOP system limits are calculated applying system/thermal hydraulics and fracture mechanics. Reference SRP Section 5.2.2; Code Case N–514; ASME Code, Appendix G, Section XI as applied in accordance with 10 CFR 50.55.	Provide setpoint curves or setpoint values.
4. The adjusted reference temperature (ART) for each reactor beltline material shall be calculated, accounting for irradiation embrittlement, in accordance with Regulatory Guide 1.99, Revision 2.	Describe the method for calculating the ART using Regulatory Guide 1.99, Revision 2.	Identify both the limiting ART values and limiting materials at the 1/4T and 3/4T locations (T=vessel beltline thickness). PWRs—identify RT_{PTS} value in accordance with 10 CFR 50.61.

REQUIREMENTS FOR METHODOLOGY AND PTLR—Continued

Provisions for methodology from administrative controls section in sts	Minimum requirements to be included in methodology	Minimum requirements to be included in PTLR
5. The limiting ART shall be incorporated into the calculation of the pressure and temperature limit curves in accordance with NUREG–0800, SRP Section 5.3.2, Pressure-Temperature Limits.	Describe the application of fracture mechanics in constructing P/T curves based on ASME Code, Appendix G, Section XI, and SRP Section 5.3.2.	Provide the P/T curves for heatup, cooldown, criticality, and hydrostatic and leak tests.
6. The minimum temperature requirements of Appendix G to 10 CFR Part 50 shall be incorporated into the pressure and temperature limit curves.	Describe how the minimum temperature requirements in Appendix G to 10 CFR Part 50 are applied to P/T curves.	Identify minimum temperatures on the P/T curves such as minimum boltup temperature and hydrotest temperature.
7. Licensees who have removed two or more capsules should compare for each surveillance material the measured increase in reference temperature (RT_{NDT}) to the predicted increase in RT_{NDT}; where the predicted increase in RT_{NDT} is based on the mean shift in RT_{NDT} plus the two standard deviation value ($2\sigma_{\Delta}$) specified in Regulatory Guide 1.99, Revision 2. If the measured value exceeds the predicted value (increase in $RT_{NDT} + 2\sigma_{\Delta}$), the licensee should provide a supplement to the PTLR to demonstrate how the results affect the approved methodology.	Describe how the data from multiple surveillance capsules are used in the ART calculation. Describe procedure if measured value exceeds predicted value. WHEN OTHER PLANT DATA ARE USED ..... 1. Identify the source(s) of data when other plant data are used. 2.a Identify by title and number the safety evaluation report that approved the use of data for the plant. Justify applicability. OR 2.b Compare licensee data with other plant data for both the radiation environments (e.g., neutron spectrum, irradiation temperature) and the surveillance test results.	Provide supplemental data and calculations of the chemistry factor in the PTLR if the surveillance data are used in the ART calculation. Evaluate the surveillance data to determine if they meet the credibility criteria in Regulatory Guide 1.99, Revision 2. Provide the results.

•Technical Specifications

The following changes must be made to the plant TS to complete the three requirements for relocating the curves and setpoints to an alternative document.

Three separate actions are necessary to modify the plant TS: (1) ''Definitions''—the addition of the definition of a named formal report (PTLR or a similar document) that would contain the explanations, figures, values, and parameters derived in accordance with an NRC-approved methodology and consistent with all of the design assumptions and stress limits for cyclic operation; (2) LCOs—the addition of references to the PTLR noting that the P/T limits shall be maintained within the limits specified in the PTLR; and (3) ''Administrative Controls''—the addition of a reporting requirement to submit to the NRC the PTLR, when it is issued, for each reactor vessel fluence period.

1. *Definitions*

Section 1.0, ''Definitions,'' should contain the following language:

Pressure Temperature Limits Report (PTLR)

The PTLR is the unit-specific document that provides the reactor vessel P/T limits and setpoints, including heatup and cooldown rates, for the current reactor vessel fluence period. These P/T limits shall be determined for each fluence period in accordance with Specification 5.X.X.X. Plant operation within these operating limits is addressed in LCO 3.X.X, ''RCS Pressure and Temperature (P/T) Limits,'' and LCO 3.X.X, ''Low Temperature Overpressure Protection (LTOP) System.''

2. *Limiting Conditions for Operation (LCOs) and Bases*

LCO 3.X.X, ''RCS Pressure and Temperature (P/T) Limits,'' and LCO 3.X.X, ''Low Temperature Overpressure Protection (LTOP) System,'' must reference the PTLR as the document where the limits and curves can be found as demonstrated in the attached model TS. The bases for these LCOs should be modified accordingly.

3. *Administrative Controls*

Section 5.X, ''Administrative Controls,'' Subsection 5.X.X, ''Reporting Requirements,'' must contain the following information:

Section 5.X.X.X *Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)*

a. RCS pressure and temperature limits for heatup, cooldown, LTOP, criticality, and hydrostatic testing as well as heatup and cooldown rates shall be established and documented in the PTLR for the following: [The individual specifications that address RCS pressure and temperature limits must be referenced here.]

b. The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in the following document(s): [Identify the NRC staff approval document(s) by date.]

c. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period and for any revision or supplement thereto.

* * * * *

Dated at Rockville, Maryland, this 25th day of May 1995.

For the Nuclear Regulatory Commission.

Brian K. Grimes,

Director, Division of Project Support, Office of Nuclear Reactor Regulation.

[FR Doc. 95–13514 Filed 6–1–95; 8:45 am]

BILLING CODE 7590–01–P

[Docket Nos. 50–295 and 50–304]

Commonwealth Edison Company, Zion Nuclear Power Station, Units 1 and 2; Issuance of Director's Decision Under 10 CFR 2.206

Notice is hereby given that the Director, Office of Nuclear Reactor Regulation, has acted on a Petition for action under 10 CFR 2.206 received from Mr. Robert K. Rutherford and 43 other security guards, dated November 3, 1994, regarding the Zion Nuclear Power Station, Units 1 and 2.

The Petitioners requested that the NRC reassess and withdraw its approval